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## **TERMINAL LEARNING OBJECTIVES:**

- 1. Describe common station blackout experiences that led to the creation of station blackout programs at commercial nuclear generation stations as defined in 10 CFR 50.63.
- 2. Examine the applicable regulatory requirements for station blackout at commercial nuclear generation stations as defined in 10 CFR 50.63.
- 3. Compare NUMARC 87-00 and NRC Reg. Guide 1.155 to assemble a comprehensive understanding of how industry guidance has shaped the NRC's expectations for station blackout.
- 4. Describe the safety implications of inadequate station blackout coping at a commercial nuclear generation station as described in 10 CFR 50.63 and NRC Reg. Guide 1.155.
- 5. Identify the scope of responsibility for engineering, systems, maintenance, and operations to identify, manage, and correct the station blackout issue given a station blackout scenario.
- 6. Identify the factors provided in NRC Reg. Guide 1.155 that are considered in determining the minimum acceptable station blackout duration capability.
- 7. Evaluate a plant's Offsite Power Design Characteristic Group based on the independence of offsite power groups, the severe weather group, and the severe weather recovery group.
- 8. Assess a plant's Emergency AC Power Configuration Group based on the redundancy of the onsite emergency AC power system.
- 9. Determine an allowed Emergency Diesel Generator (EDG) reliability based on the calculated EDG reliability.
- 10. Select a unit's minimum required station blackout coping duration based on the Emergency AC Power Group, Offsite Power Group, and Allowed EDG Target Reliability.
- 11. Describe the necessary considerations for determining a plant's capability to cope with a station blackout.
- 12. Calculate the available and required condensate inventory to maintain adequate reactor coolant system inventory for decay heat removal.
- 13. Evaluate the capacity of the Class 1E batteries to ensure their adequacy in supporting decay heat removal and monitoring during the station blackout coping duration.
- 14. Assess that all Air-Operated Valves (AOVs) required for decay heat removal have an adequate supply of reserve air or can be opened manually during a station blackout.
- 15. Identify the Dominant Areas of Concern (DAC) and acceptance criteria that may be applied for loss of ventilation to those areas.
- 16. Evaluate the design adequacy and capability of equipment necessary for safe shutdown for increased temperatures due to a loss of ventilation during a station blackout.
- 17. Analyze the ability of the plant to maintain appropriate containment integrity for the required coping duration during a station blackout event.

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- 18. Describe the requirements for station blackout for an Alternate AC power source based on the guidance of NRC Reg. Guide 1.155 and NUMARC 87-00.
- 19. Identify potential modifications to the plant that may be necessary to extend the time the plant can cope with a station blackout in case the minimum required coping time is not met.
- 20. Describe the Extended Loss of AC Power (ELAP) event at Fukushima Daiichi that led to the issuance of EA-12-049 and JLD-ISG-2012-01, requiring mitigating strategies for beyond-design-basis external events.
- 21. Review the industry guidance provided in NEI 12-06, Diverse and Flexible Coping Strategies (FLEX) Implementation Guidance.
- 22. Identify the different phases of FLEX and the equipment used to support each phase.
- 23. Evaluate how FLEX strategies to extend or recover power will mitigate an ELAP event.
- 24. Define the analyses required to support FLEX strategy development.
- 25. Describe the difference between station blackout coping and FLEX strategies.
- 26. Explain the procedural approach used to implement FLEX strategies.

## **KEY INDUSTRY DOCUMENTS:**

## **Publicly Available Documents**

- 1. 10 CFR 50.63, "Loss of all alternating current power"
- 2. NRC Regulatory Guide 1.155 (ML003740034), "Station Blackout"
- 3. NUMARC 87-00 (ML12137A732), "Guidelines and Technical Bases for NUMARC initiatives addressing Station Blackout at Light Water Reactors"
- NUREG-1032, "Evaluation of Station Blackout Accidents at Nuclear Power Plants, Technical Findings Related to Unresolvec Safety Issue A-44"
- 5. NUREG-1410, "Loss of Vital AC Power and the Residual Heat Removal System During Mid-Loop Operations at Vogtle Unit 1 on March 20, 1990"
- 6. NUREG/CR-5496, "Evaluation of Loss of Offsite Power Events at Nuclear Power Plants: 1980 1996"
- 7. NUREG/CR-6890, "Reevaluation of Station Blackout Risk at Nuclear Power Plants: Analysis of Station Blackout Risk"
- 8. NUREG-0800, "U.S. Nuclear Regulatory Commission Standard Review Plan"
- 9. NRC EA-12-049 (ML12054A735), "Issuance of Order to Modify Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events"
- JLD-ISG-2012-01 (ML17005A188), "Compliance with Order EA-12-049, Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events", Revision 2
- 11. NUREG-1776, "Regulatory Effectiveness of the Station Blackout Rule"
- 12. NEI 12-06, "Diverse and Flexible Coping Strategies (FLEX) Implementation Guide"
- 13. Regulatory Basis Document NRC-2011-0299, "Station Blackout Mitigation Strategies," ADAMS Accession Number ML13171A061
- 14. Exelon Generation Company, "Report of Full Compliance with March 12, 2012 Commission Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events (order Number EA-12-049)"
- 15. "Virginia Electric and Power Company (Dominion) Surry Power Station Units 1 and 2 Compliance Letter and Final Integrated Plan in Response to the March 12, 2012 Commission Order Modifying Licenses with Regard to Requirements for Mitigating Strategies for Beyond-Design-Basis External Events (Order Number EA-12-049)," ADAMS Accession No. MI 16033A353

## **INPO Documents**

- 16. INPO Event Report 13-10, "Nuclear Accident at the Fukushima Daiichi Nuclear Power Station"
- 17. INPO Event Report 12-78, "Station Blackout and Loss of Shutdown Cooling Event Resulting from Inadequate Risk Assessment"